

Opportunities in Materials Science and Engineering Driven by ICF Hybrids – Materials for LIFE

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Hybrid reactor systems are once again being considered as a possible means of producing electrical power from natural or depleted uranium, uranium of various enrichments, plutonium, spent nuclear fuel, and thorium, without the need for isotopic enrichment. Ultimately, such technology might enable the more complete burning of spent nuclear fuel from light water reactors, without chemical separation into weapons-attractive actinide streams.

The National Ignition Facility (NIF) is complete and operational, and initial experiments that are expected to lead to ignition are well underway with rapid progress being made. The Department of Energy's NIF is now a reality, and ignition is visible on the horizon. Once ignition is achieved, the Nation's scientific and technical community will have a unique opportunity, the investigation of how to best leverage the ignition event to help solve our problems related to energy independence, global warming, and the eventual disposal of our inventory of spent nuclear fuel.

In the case of LIFE (Laser Inertial Fusion Energy), an ICF-type hybrid that will leverage ignition, a point-source of high-energy neutrons produced by laser-generated, thermonuclear fusion within a target will be used to achieve ultra-deep burn-up of the fertile or fissile fuel in a sub-critical fission blanket. Starting from as little as 300 to 500 MW of fusion power, this machine could conceivably generate 2000 to 3000 MW of thermal energy, and over 1000 MW of electrical energy. Of course, hydrogen could also be generated in a variety of ways.

Based upon preliminary design work, it is now believed that ICF-type hybrids could be developed to the point where they could help meet worldwide electricity need in a safe and sustainable manner, while shrinking the stockpile of spent nuclear fuel and excess weapons materials. Furthermore, it appears that such hybrid systems could reduce the energy-normalized load on any future geological repository for spent nuclear fuel and high-level waste by a ~20X.

The development of modularized ICF-type hybrids will serve as an important driver for advanced materials science and development in Silicon Valley, and throughout the Nation's research establishment. Materials for fusion-fission hybrid reactors fall into several broad categories, including: (1) lasers and optics, (2) fusion targets, (3) first wall, (4) structural material, (5) neutron multiplication blanket, (6) fission blanket, (7) reflector, (8) coolant, and (9) balance of plant. Issues related to lasers and optics, as well as fusion targets are discussed elsewhere. The first wall and structural materials in such a system must be able to withstand relatively high temperatures, intense neutron bombardment, and corrosive

attack by molten fluoride salts. The high power density may require molten fluoride salt coolants, such as FLiBe (Li_2BeF_4) or FLiNaK (Li-Na-K-F), and perhaps liquid metals. Materials under consideration for these applications include refractory metals and alloys, oxide dispersion strengthened steels, and high-temperature composites. Materials and conditions for one hypothetical design are shown in Figure 1.

Fuels for the fission blanket must be able to contain a relatively large amount of fission gas, as well as stresses due to the volumetric changes that accompany transmutation. Several possible fuel forms have been identified for investigation for a wide range of promising applications, including but not limited to an enhanced TRI-structural-ISO-tropic (TRISO) fuel; solid hollow core (SHC) fuels; metallic and ceramic inert matrix fuels (IMFs); and molten salt fuels (MSFs). The neutron multiplier could be made of metallic beryllium, beryllides which may be more radiation resistant, or other appropriate materials. Materials and fuels will have to be selected, and developed as necessary to enable operation in these harsh conditions. While several possible fission fuels are being considered, published data indicates that reasonably high burn-ups may be possible with enriched uranium in TRISO particles. However, to achieve comparable burn-up with natural or depleted uranium, substantially more radiation damage will have to be tolerated.

In summary, once ignition is achieved, the Nation's scientific and technical community will have a unique opportunity, the investigation of how to best leverage the ignition event to help solve our problems related to energy independence, global warming, and the eventual disposal of our inventory of spent nuclear fuel. Most of the materials that will be developed in support of such hybrid systems will have multiple uses, spanning from optoelectronics to nuclear energy, aerospace and transportation.

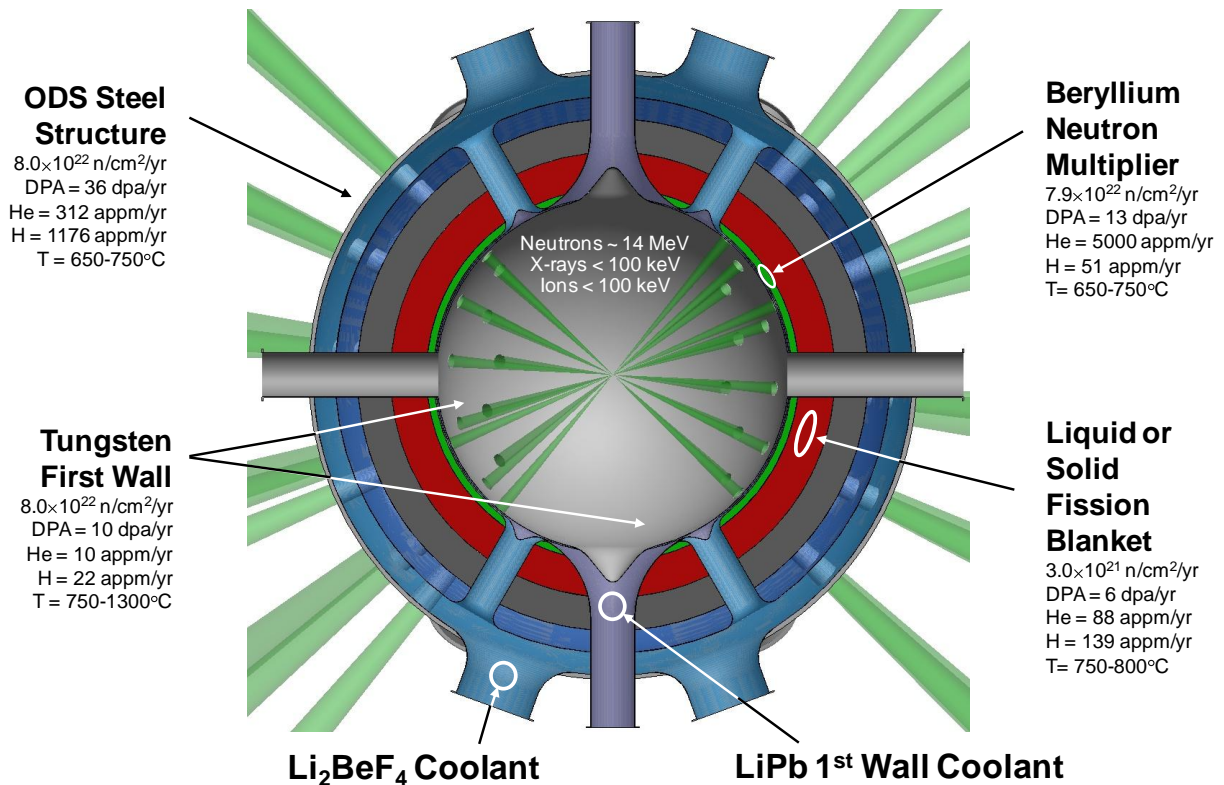


Figure 1 – Conditions challenging materials and fuels in hypothetical ICF-type hybrid system.

Bibliography

J. C. Farmer, LIFE Materials: Overview of Fuels and Structural Materials Issues, Volume 1, LLNL-TR-407386 Rev. 2c, February 7th 2009, Lawrence Livermore National Laboratory (DOE Contract DE-AC52-07NA27344), 7000 East Avenue, Livermore CA, 180 p.

T. R. Allen, J. Gan, J. I. Cole, S. Ukai, S. Shutthanandan and S. Thevuthasan: The stability of 9Cr-ODS oxide particles under heavy-ion irradiation, *Nuclear Science & Engineering* 151 (2005) 305–312.

T. Angeliu, J. Ward and J. Witter: Assessing the effects of radiation damage on Ni-base alloys for the Prometheus space reactor system (LM-06K033, April 4th 2007, Knolls Atomic Power Laboratory, KAPL, Lockheed Martin, P. O. Box 1072, Schenectady, New York 12301-1072, Telephone 518-395-6163, Email angeltm@kapl.gov).

E. T. Cheng, B. J. Merrill and Dai-Kai Sze: Nuclear aspects of molten salt blankets, *Fusion Engineering and Design*, *Fusion Engineering and Design* 69 (2003) 205-213.

K. F. Farrell and E. H. Lee: Ion damage in a Fe-10Cr-6Mo-0.5Nb ferritic steel, *Radiation-Induced Changes in Microstructure, 13th International Symposium, Part I*, Garner, Packan, Kumar, Editors, ASTM STP 955 (American Society for Testing and Materials, ASTM, Philadelphia, Pennsylvania, 1986) pp. 498-519.

M. A. DeLuchi: Hydrogen vehicles: an evaluation of fuel storage, performance, safety, environmental impacts, and cost, *International Journal of Hydrogen Energy* 14, 2 (1989) 81-130.

A. R. Foster, R. L. Wright, Jr.: Chapter 4, Nuclear Reactions, Fusion, Chapter 11, Radiation Damage and Reactor Materials, Dispersion-Type Alloys (TRISO), Chapter 13, Nuclear Reactors, Molten Salt Breeder Reactor, Fusion by Laser, Fusion-Fission Symbiosis, Table 13.6, Typical MSBR Compositions and Properties, Figure 13.31, MSBR Fuel Processing Flow Diagram, *Basic Nuclear Engineering, 2nd Ed.* (Allyn and Bacon, Incorporated, Boston, MA, 1973) pp. 72-80, 330-342, 416-422, 446-449.

W. R. Grimes, S. Cantor: Molten salt as blanket fluids in controlled fusion reactors, *The Chemistry of Fusion Technology*, Plenum Press, 1972.

C. R. Hammond: The Elements, Nomenclature of Inorganic Chemistry, Physical Constants of Inorganic Compounds, *Chemical Rubber Company (CRC) Handbook of Chemistry and Physics, 61st Ed.* (R. C. Weast, M. J. Astle, Editors, CRC Press Incorporated, Boca Raton, FL 33431, 1980) B-2 through B166.

J. I. Han and J. -Y. Lee: Hydriding kinetics of LaNi₅ and LaNi_{4.7}Al_{0.3}, *International Journal of Hydrogen Energy* 14, 3 (1989) 181-186.

P. N. Haubenreich, J. R. Engel: Experience with the molten-salt reactor experiment, *Nuclear Applications and Technology* 8 (1970) 118-121.

J. E. Indacochea, J. L. Smith, K. R. Litko and E. J. Karel: Corrosion performance of ferrous and refractory metals in molten salts under reducing conditions, *Journal of Materials Research* 14, 5 (1999) 1990-1995.

R. L. Klueh, N. Hashimoto and P. J. Maziasz: New nano-particle-strengthened ferritic-martensitic steels by conventional thermo-mechanical treatment, *Journal of Nuclear Materials* 367-370 (2007) 48-53.

R. J. Lauf, T. B. Lindemer and R. L. Pearson: Out-of-reactor studies of fission-product/silicon-carbide interactions in HTGR fuel particles, *Journal of Nuclear Materials* 120 (1984) 6-30.

B. J. Makenas, R. G. Trenchard, S. L. Hecht, J. M. McCarthy and F. A. Garner: The effect of swelling in Inconel 600 on the performance of FFTF reflector assemblies, *Radiation-Induced Changes in Microstructure, 13th International Symposium, Part I*, Garner, Packan, Kumar, Editors, ASTM STP 955 (American Society for Testing and Materials, ASTM, Philadelphia, Pennsylvania, 1986) pp. 206-229.

R. W. Moir, J. D. Lee, F. J. Fulton, F. Huegel, W. S. Neef, Jr., A. E. Sherwood, P. H. Berwald, R. H. Whitley, C. D. C. Wong, J. H. DeVan, W. R. Grimes, S. K. Ghose: Design of a Helium-Cooled Molten-Salt Fusion Breeder, *Fusion Technology* 8 (1985) 465-473.

H. Nishimura, A. Suzuki, T. Terai, M. Yamawaki, S. Tanaka, A. Sagra and O. Motojima: Chemical behavior of Li_2BeF_4 molten salt as a liquid tritium breeder, *Fusion Engineering and Design* 58-59 (2001) 667-672.

J. A. Ober: Lithium, *2006 Minerals Yearbook; Mineral Commodity Summaries* (USGS, October 2007, Tel 703-648-7717, Email jobber@usgs.gov).

D. O'Donnell: Joining of Oxide Dispersion Strengthened Materials, *Special Welding and Joining Topics*, pp. 1037-1040.

H. Okamoto: Pd-Si Phase Diagram, *Desk Handbook of Phase Diagrams for Binary Diagrams* (American Society of Metals, International, Materials Park, Ohio, 2000) p. 658.

L. Pauling: *General Chemistry* (Dover Publications, New York, NY, 1947, 1950, 1970) 959 p.

D. Petti, P. Martin, M. Phélip and R. Ballinger: Development of improved models and designs for coated-particle gas reactor fuels (Final Report, International Nuclear Energy Research Initiative (INEEL/EXT-05-02615, Idaho National Engineering and Environmental Laboratory, Idaho Falls, Idaho 83415, December 2004).

D. A. Petti, G. R. Smolik, M. F. Simpson, J. P. Sharpe, R. A. Andrerl, S. Fukada, Y. Hatano, M. Hara, Y. Oya, T. Terai, D-K. Sze, S. Tanaka: Jupiter-II molten salt FLIBE research, an update on tritium, mobilization and redox chemistry experiments, *Fusion Engineering and Design* 81 (2006) 1439-1449.

B. A. Pint, P. F. Tortorelli, A. Jankowski, J. Hayes, T. Muroga, A. Suzuki, O. I. Yeliseyeva, V. M. Chernov: Recent progress in the development of electrically insulating coatings for a liquid lithium blanket, *Journal of Nuclear Materials* 329-333 (2004) 119-124.

B. A. Pint, J. L. Moser and P. F. Tortorelli: Liquid metal compatibility issues for test blanket modules, *Fusion Engineering and Design* 81 (2006) 901-908.

M. W. Rosenthal, P. R. Kasten, R. B. Briggs: *Nuclear Technology* 8, 2 (1970) 111.

A. I. Ryazanov, A. V. Klaptsov, A. Kohyama and H. Kishimoto: Radiation swelling of SiC under neutron irradiation, *Journal of Nuclear Materials* 307-311 (2002) 1107-1111.

- R. Schaublin et al.: Microstructural development under irradiation in European ODS FM steels, *Journal of Nuclear Materials* 351 (2006) 247-260.
- K. B. Shedd, Beryllium, *2006 Minerals Yearbook; Mineral Commodity Summaries* (USGS, October 2007, Tel 703-648-4974, Email kshedd@usgs.gov).
- L. Smart and E. Moore: *Solid State Chemistry, An Introduction*, 2nd Ed. (Chapman and Hall, London, UK, 1992, 1996; Reprinted by Stanley Thornes Publishers Ltd., Ellenborough House, Wellington Street, Cheltenham GL50 1YW, UK) 379.
- D. L. Smith, M. C. Billone, K. Natesan: Vanadium-base alloys for fusion first-wall/blanket applications, *International Journal of Refractory Metals and Hard Materials* 18 (2000) 213-224.
- L. L. Snead, T. Nozawa, Y. Katoh, T-S. Byun, S. Kondo and D. A. Petti: Handbook of SiC properties on fuel performance modeling, *Journal of Nuclear Materials* 371 (2007) 329-377.
- A. Suzuki, T. Muroga, B. A. Pint, T. Yoneoka and S. Tanaka: Corrosion behavior of AlN for self-cooled Li/V blanket application, *Fusion Engineering and Design* 69 (2003) 397-401.
- I. N. Sviatoslavsky, M. E. Sawan, E. A. Mogahed, S. Majumdar, R. Mattas, S. Malang, P. J. Fogarty, M. Friend, C. P. C. Wong and S. Sharafat: Engineering and geometric aspects of the solid wall re-circulating fluid blanket based on advanced ferritic steel, *Fusion Engineering and Design* 72 (2004) 307-326.
- L. Theodore: Energy transport, Example 4.6.2, Long hollow cylinder, Example 4.6.3, Solid cylinder with uniform heat generation rate, *Transport Phenomena for Engineers* (International Textbook Company, London, UK, 1971) pp. 157-161.
- P. A. Thornton and V. J. Colangelo: *Fundamentals of Engineering Materials* (Prentice-Hall Incorporated, Englewood Cliffs, NJ 07632) 679 p.
- R. Treseder, R. Baboian, and C. Munger: Polarization resistance method for determining corrosion rates, *Corrosion Engineer's Reference Book*, 2nd Ed. (National Association of Corrosion Engineers, NACE, Houston, Texas, 1991) pp. 65-66.
- D. F. Williams, L. M. Toth and K. T. Clarno: Assessment of candidate molten salt coolants for the advanced high-temperature reactor (ORNL/TM-2006/12, Nuclear Science and Technology Division, Oak Ridge National Laboratory, P.O. Box 2008, Oak Ridge, Tennessee 37831-6283, March 2006).
- I. Wright, B. Pint and Z. Ping Lu: Overview of ODS alloy development (Oak Ridge National Laboratory, 1 Bethel Road, Oak Ridge, TN).
- S. Ukai, M. Fujiwara: Perspective of ODS alloys application in nuclear environments, *Journal of Nuclear Materials* 307-311 (2002) 749-757.
- S. Ukai et al.: Oxide dispersion strengthened (ODS) fuel pin fabrication for BOR-60 irradiation test, *Journal of Nuclear Science & Technology* 42, 1 (2005) 109-122.
- S. Ukai and S. Ohtsuka: Low cycle fatigue properties of ODS ferritic-martensitic steels at high temperature, *Journal of Nuclear Materials* 367-370 (2007) 234-238.

I. Ursu: Chapter 5, Moderator Materials, Chapter 12, Materials in Fusion Reactors, *Physics and Technology of Nuclear Materials* (Pergamon Press, New York, New York, 1985) pp. 169-206, 406-457.

W. C. Young: Chapter 12, Shells of Revolution; Pressure Vessels; Pipes, Table 32, Spherical Vessel, Case No. 2a, Uniform Internal Pressure, *Roark's Formulas for Stress and Strain, 6th Ed.* (McGraw-Hill Incorporated, San Francisco, CA) pp. 515-646.

S. J. Zinkle and N. M. Ghoniem: Operating temperature windows for fusion reactor structural materials, *Fusion Engineering and Design*, 51-51 (2000) 55-71.

S. J. Zinkle: Materials in extreme nuclear environments (Invited Presentation, National Ignition Facility, Lawrence Livermore National Laboratory, March 17, 2008).